

STUDY OF ABNORMAL HEAT TRANSFER DURING FORCED AND NATURAL CONVECTION SCENARIOS IN A PRISMATIC CORE OF A VHTR: NUMERICAL AND EXPERIMENTAL RESULTS

Francisco I. Valentín^a, Narbeh Artoun^a, Ryan Anderson^b and Masahiro Kawaji^{a,c,*}

^aCity College of New York

^bMontana State University

^cThe CUNY Energy Institute

fivalente@gmail.com; narbeh.artoun@gmail.com; ryan.anderson@coe.montana.edu;

*Corresponding Author: kawaji@me.ccnycunyu.edu

ABSTRACT

Very High Temperature Reactors (VHTRs) are one of the Generation IV gas-cooled reactor models proposed for implementation in next generation nuclear power plants. A high temperature/pressure test facility for forced and natural circulation experiments has been constructed. This test facility consists of a single flow channel in a 2.7 m (9') long graphite column equipped with four 2.3kW heaters. Extensive 3D numerical modeling provides a detailed analysis of the thermalhydraulic behavior under steady-state, transient, and accident scenarios. In addition, forced convection experiments with air and nitrogen were conducted for inlet Reynolds numbers from 1,500 to 70,000. Numerical results, using commercial finite element package, COMSOL Multiphysics, were validated with forced convection data displaying maximum percentage errors under 15%. Based on this agreement, simulations are also extended to study natural circulation of nitrogen and helium between two connected vessels, resembling the behavior between inner and outer cooling channels in a VHTR reactor core. Results were obtained for Rayleigh numbers of $\sim 10^6$. This work also examines flow laminarization for a full range of Reynolds numbers including laminar, transition and turbulent flow under forced convection and natural circulation and its impact on heat transfer under various scenarios to examine the thermal-hydraulic phenomena that could occur during both normal operation and accident conditions.

KEYWORDS

Forced convection, heat transfer, VHTR, high temperature gas reactor

1. BACKGROUND

Very High Temperature Reactors (VHTRs) are one of the Generation IV reactors that have been proposed for DOE's Next Generation Nuclear Plant. Several characteristics have stimulated this development including higher efficiencies due to higher working temperatures and the additional benefits of passive safety systems including intracore conduction and natural circulation for decay heat removal and dissipation in accident scenarios. Key phenomena have also been identified as leading to localized hot spots in the nuclear reactor, including: degraded heat transfer in coolant channels, laminarization of flow, effects of bypass flow, and non-uniform heat generation across the core.^[1-2]

One issue in studying flow laminarization is implementing applicable turbulence models with appropriate parameters. Mikielewics et al.^[3] successfully studied internal velocity and temperature fields of strongly heated internal flows numerically. They examined eleven turbulence models, requiring two partial differential equations or less. They concluded that the model by Launder^[4] performed best in predicting experimental data.^[3] Their study highlights the difficulty involved in modeling even simple axisymmetric gas flows in pipes subjected to intense heating, which take into account variable gas properties since only one out of the eleven models agreed on heat transfer parameters such as the Nusselt number. Several of these other $k-\epsilon$ models specifically developed for low Reynolds number applications even gave unacceptably high Nusselt number predictions.^[3] In addition, many of these studies were specific to the conditions used in the experiments of Shehata and McEligot.^[5,6], thus the knowledge base available outside the conditions used in Shehata and McEligot's experiments is limited. This points to the limited range of applications of even the successful models, since their validation data was constrained to small tubes, low

4. The fluid flow model (laminar, $k-\varepsilon$ and low Re $k-\varepsilon$ turbulence models) used in numerical simulations had a minor impact on the Reynolds number predictions (<1.5% difference) but as much as a 15% difference in the Nusselt number predictions.

Future work includes further CFD model validation based on experimental data and carrying simulations for helium as the working fluid.

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